

AFCI Separations Activities

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Presented at:
Joint ANS/AIChE Local Section Meeting
Knoxville, TN
February 10, 2009

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- Separations Campaign Overview
- Key Campaign Accomplishment / Goals
- Coupled End to End (CETE) Project
- Off-gas Treatment
 - ORNL
 - INL
 - SNL
- Centrifugal Contactors
 - INL





Separations Campaign Program Overview

Major program elements

- Advanced aqueous separations
- Advanced electrochemical separations
- Process equipment scale-up and development
- Off-gas treatment
- Process control and monitoring
- Process modeling and simulation
- EBR-II spent fuel treatment
- Separations regulatory and safety crosscut







Separations Campaign FY-08 Major Accomplishments

- Revised campaign research strategy for aqueous processing
 - De-emphasize repetitive testing of separation process at lab-scale using small amounts of used nuclear fuel
 - Focus on head-end operations
 - Voloxidation to separate tritium
 - Off-gas capture and immobilization of ¹²⁹I, ⁸⁵Kr (and Xe), ³H, ¹⁴C
 - Demonstrate product conversion using modified direct denitration
 - Transition development activities from laboratory testing to engineering development
 - Incorporate industry input into campaign development activities
 - Increase Technology Readiness Levels (TRL's) > 6
 - Incorporate solvent recycle effects into testing
 - Integrate Waste Form development with Separations development





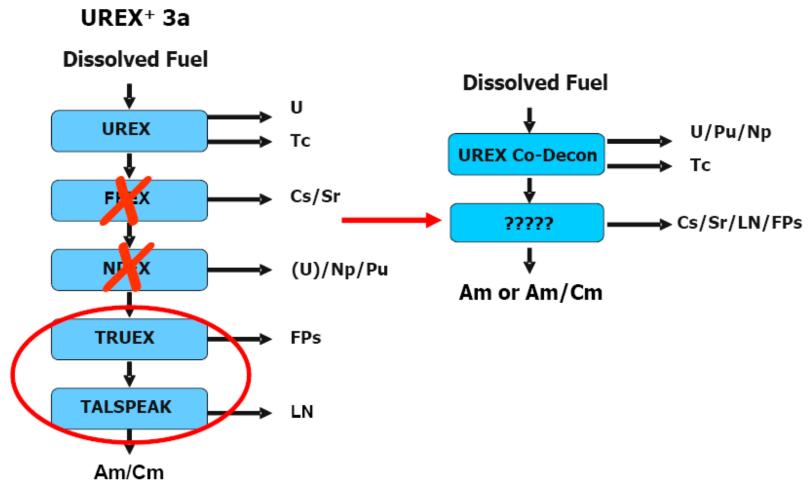
Separations Campaign FY-08 Major Accomplishments

- Revised campaign research strategy for aqueous processing
 - Focus on simplification of separation processes
 - Decay store HLW rather than separate Cs/Sr
 - Reduce number of separation steps and separated products
 - Separate U/Pu/Np together by manipulating valance states
 - Combine TRUEX/TALSPEAK or use alternative technology
 - Avoid dependence on rigid pH control
 - Develop science-based approach to develop alternative Am/Cm separation process
 - Formation of "SIGMA TEAM"
 - Multi-lab team focused on development of robust separation process for separation of Am or Am/Cm in a single step





Separations strategy evolution







Selected Development Focus Areas for Aqueous Technologies

- Focus on simplification of process is key objective to improve operation and reliability and reduce cost
 - Studies have shown that the UREX+ separations options are complicated and costly
 - Largest challenge is separation of Am/Cm or Am alone
 - Focus area for Sigma Team
- CETE Run 2 (focus on off-gas capture and MDD)
- Capture and immobilization of volatile off-gas (1291, 85Kr, 3H, 14C?)
 - Voloxidation to separate tritium
 - Off-gas capture and immobilization
- Testing at scales larger than laboratory scale, in prototypic equipment w/ solvent recycle (increase TRL's)
- Integration of separations and waste form development activities (each affects the other)
- Process monitoring and control improvements
- Collaboration with industrial teams to identify and resolve design data needs





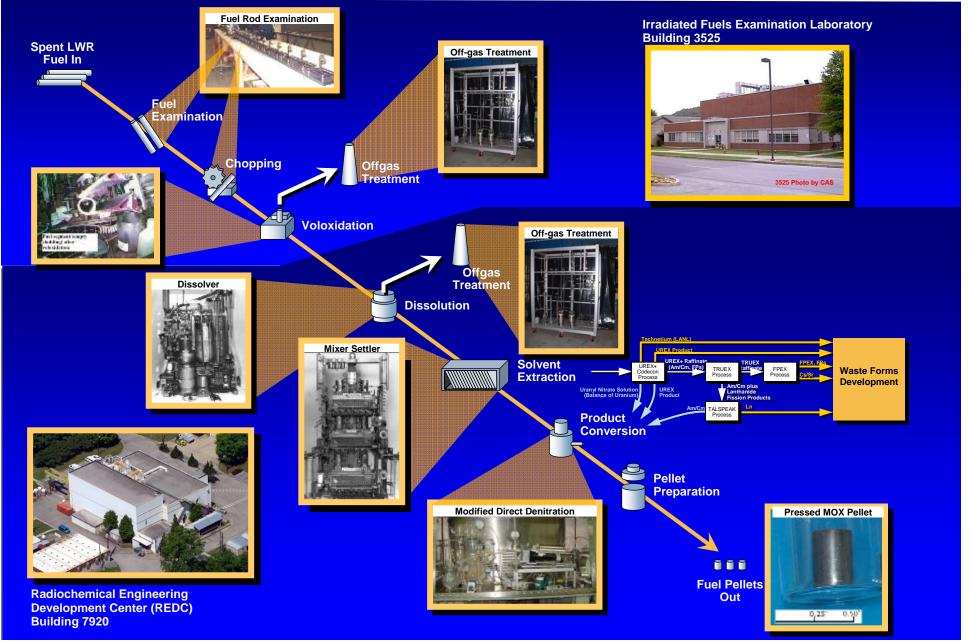
Introduction to the CETE Demonstration

- Develop/Demonstrate Advanced Recycling Technologies
- Multiple process runs ~ 5 10 kgs/yr of SNF
- Identify and Resolve Scientific & Technical Uncertainties
 - Interfacial Issues
 - Process Robustness
- Provide Research Products for Evaluation Across DOE Complex
 - Actinides
 - FP Waste form
- FY09: Focus on Head End Processing and Product Conversions





Coupled-End-to-End Demonstration Overview

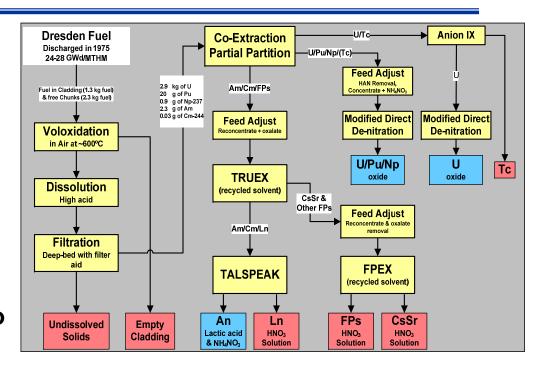




Highlights and Lessons Learned from CETE Run 1

Flowsheet Demonstration

- Partial partitioning of a U-Pu-Np Product: No separated Pu
- TRUEX-TALSPEAK for Minor Actinide Separation
- FPEX for Cs/Sr Separation
- Converted U-Pu-Np and U by Modified Direct Denitration
- Demonstrated Fabrication U-Pu-Np co-converted product to pellets.



- Removal and disposal or recycle of the residual chemical complexants (DTPA/Lactic Acid) used in the TRUEX-TALSPEAK processes from the americium-curium product is difficult.
 - Recovery by re-extraction into HDEHP has been successful





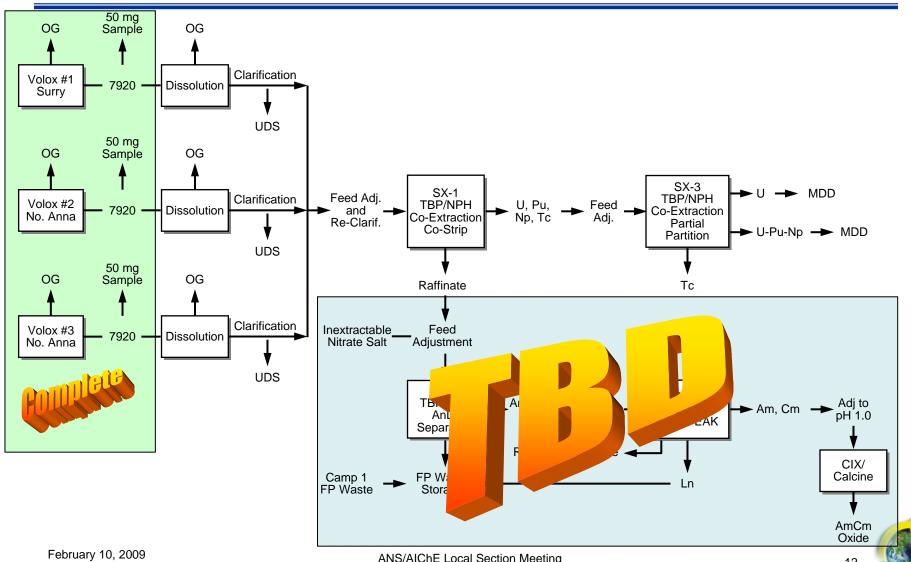
Product Conversions

- Modified Direct Denitration shows promise for simplifying the interface between separation and fuel fabrication.
- Produces a powder with good ceramic properties for pellet fabrication.
- Further R&D required
 - Process development
 - Scaleup
 - Qualifying the ceramic product





CETE Run 2 Plans



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Management of Volatile Fission Products

Volatile components that must be considered have a wide range of half-lives and disposal requirements:

3H
 12.31 yr
 14C
 5715 yr

Xe/85Kr
 Stable/10.76 yr

- ¹²⁹I 1.57 x 10⁷ yr

- Regulatory Drivers, some not written with consideration for fullscale reprocessing facilities, unlikely to be relaxed
- Data still needed on process integration and removal process interactions
 - Description of integrated capture systems for CETE Demo
 - Complemented by cold Bench-scale testing at INL
 - Waste form work for Iodine at SNL





Federal regulations limit radionuclide air emissions in the U.S.

	Dose equivalent to public, mrem/yr	Max fuel cycle emissions per GWyr energy produced	Ambient air conc at site boundary, Ci/m ³
DOE facilities (40CFR61.92)	10		
Nuclear fuel cycle (40CFR190.10)	25 (75 to thyroid, 25 to any other organ)	⁸⁵ Kr: <50,000 Ci ¹²⁹ I: <5 mCi ²³⁹ Pu: <0.5 mCi	
NRC licensees (10CFR20.1301, .1302, App. B)	100		³ H: 1.0E-7 ¹⁴ C as CO ₂ : 3.0E-7 ⁸⁵ Kr: 7.0E-7 ¹²⁹ I: 4.0E-11
Others (40CFR61.102)	10 (3 from ¹²⁹ l alone)		





40 CFR 190 Derived Process Requirements

Isotope	Ci/MTIHM	Ci/GW(e)-yr	Min Required DF
129	0.02648	0.89	178
⁸⁵ Kr (5 year cooled)	7121	239,000	4.77
85Kr (10 year cooled)	5154	173,000	3.45
85Kr (30 year cooled)	1414	47,000	0.95

Note: Burn-up: 33 GWd/MTIHM

Isotope	Ci/MTIHM	Ci/GW(e)-yr	Min Required DF
129	0.04149	0.83	167
⁸⁵ Kr (5 year cooled)	11620	234,000	4.67
⁸⁵ Kr (10 year cooled)	8413	169,000	2.38
⁸⁵ Kr (30 year cooled)	2309	46,000	0.93

Note: Burn-up: 55 GWd/MTIHM

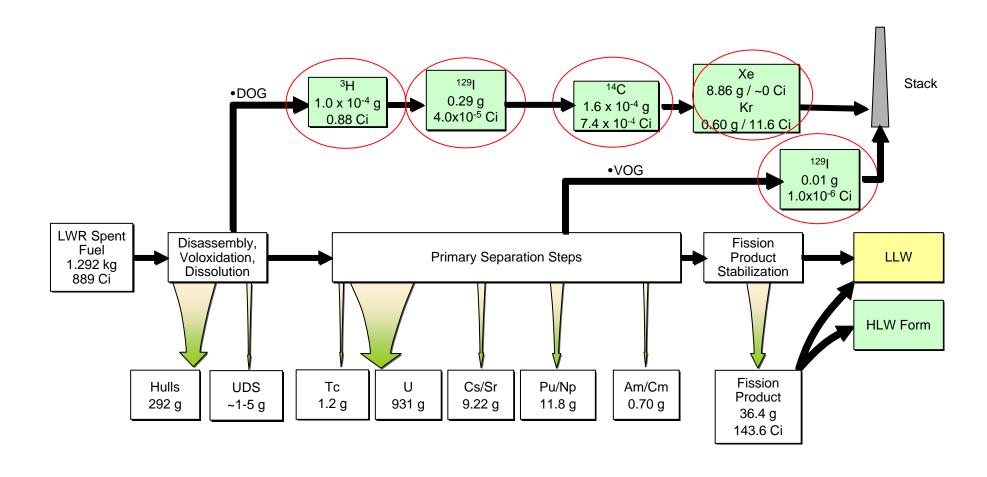
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Simple Reprocessing Demonstration

(Mass Basis: 1 kg SNF; 55 GWD/MTIHM; 5 year Cooling)

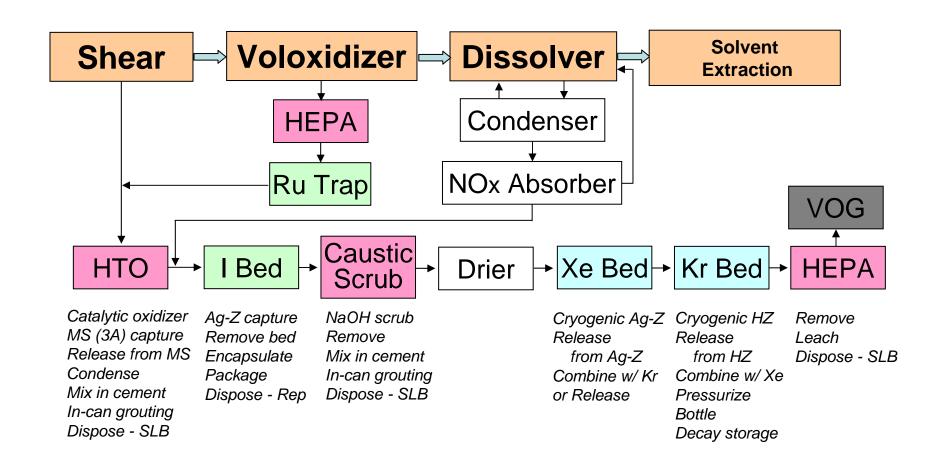




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Generic Head-End Off-Gas Treatment Concept







FY 09 CETE Based Tests (ORNL)

Hot Integrated Off-gas capture tests on the Voloxidation and Dissolver Off-Gas streams

- Objectives
 - Close volatile component material balances
 - Voloxidation
 - Dissolution
 - Analysis for residual iodine in dissolver solution
 - Shearing of full length fuel Run 3 or later
 - Understand impacts of head-end processing conditions on volatile component releases
 - Determine capture process interactions





Voloxidation Run 2—Plan and Status

- Voloxidation processing for Run 2 was planned as three batches
 - Batch sizes of 1 to 2 kg of spent fuel
 - Total production of at least 5 kg to support separations activities
 - Different conditions were planned for each batch to obtain information on reaction properties and subsequent dissolution properties of product powder (e.g. undissolved solids)
- Flexibility was built into the test plan to permit selection of alternative fuel materials, depending upon availability

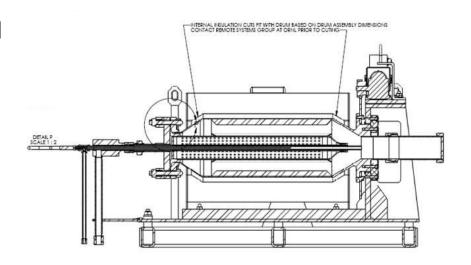
Batch	Fuel Source	Fuel Amt (kg)	Oxidant	Temperature (°C)
1	Surry-2	1–2	Air	500.
2	North Anna	2	Air	600.
3	North Anna	2	oxygen	550.

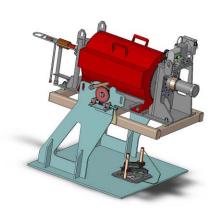


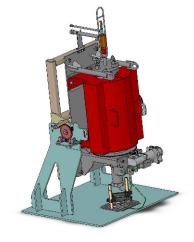


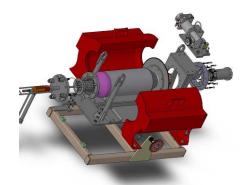
Small Scale Voloxidation

- Most of the retort tube is enveloped by the furnace
- Rotating tube
- Variable operating environment
- Removable Hulls basket
- Powder can integrated into design
- Tilting platform can force material into hotter zones

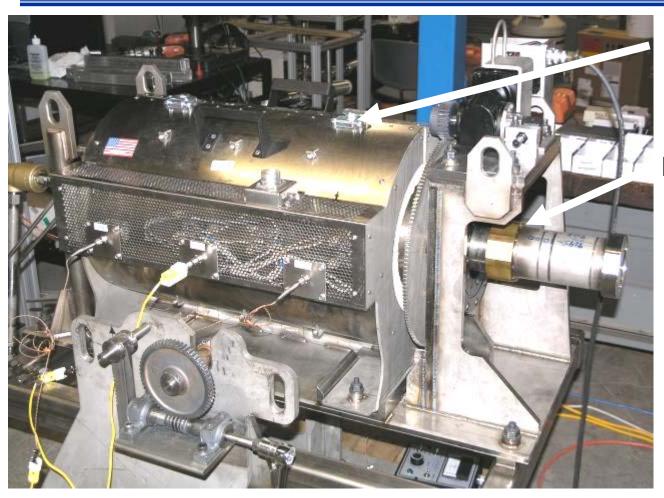












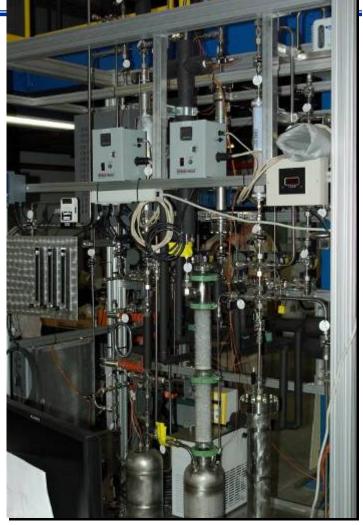
Rotary Kiln w/ Heater in Place

Product Canister



Voloxidizer Off-Gas Rack









Voloxidation Run 2—Batch 1

First batch was comprised of Surry-2 fuel

- Initial enrichment 3.11%
- Burnup: 36 GWd/MT heavy metal
- Cooling time: 27 years (discharged from reactor in 1981)
- Amount: 1704 g (316 g hulls, 1388 g fuel)

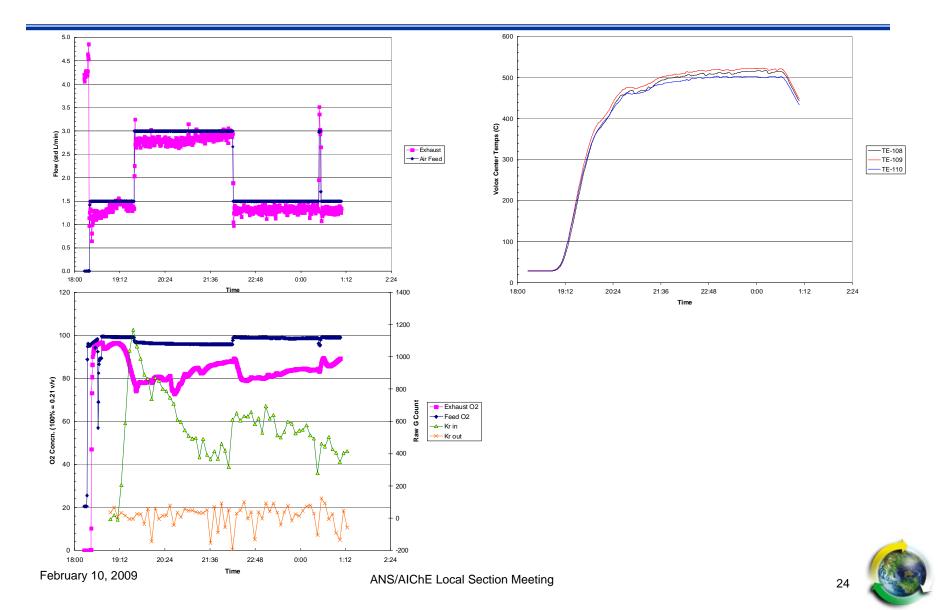
Experiment

- Fuel was oxidized in air at about 500°C
- The run was prematurely terminated due to apparent cessation of oxygen consumption
 - Occurred after 4 hours of operation at temperature
 - Weighing of product powder and hulls showed that fuel was only 70% oxidized
 - There was potential for a cold zone near the closure flange, causing slow kinetics
- Complete batch of fuel was re-run to finish the oxidation
 - Operation at temperature for 4 hours was done to ensure complete oxidation
 - 1542 g fuel powder recovered (indicates DU carry-over from shakedown run)
 - The voloxidizer was periodically tilted from the horizontal to cause fuel to migrate from the cold zone into the hot furnace zone
 - Both oxygen consumption and ⁸⁵Kr evolution was used to monitor progress and reaction endpoint



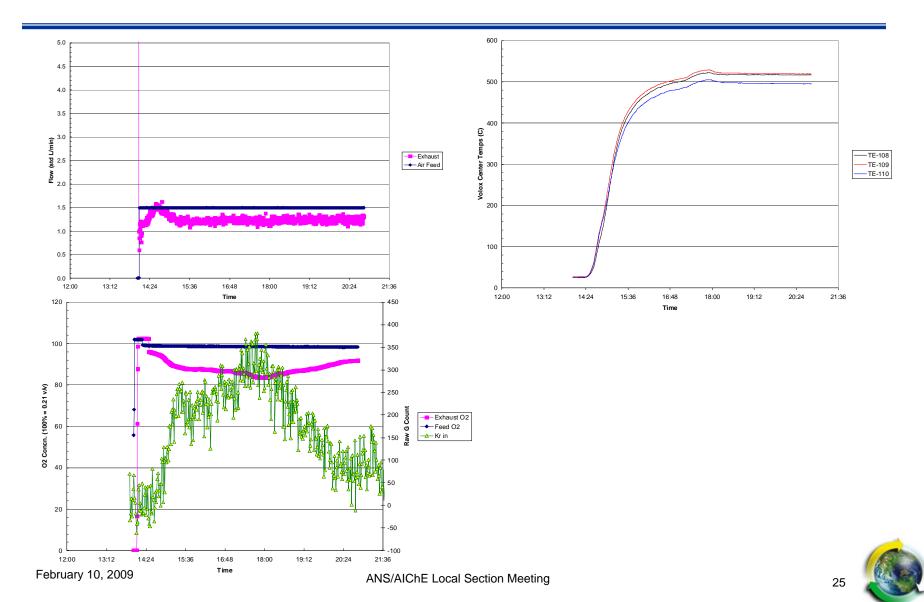


Run 2—Batch 1—Part A O₂ Concentration, Air Flow, and Temperatures





Run 2—Batch 1—Part B O₂ Concentration, Air Flow, and Temperatures





Summary of INL Off-Gas Work

■ FY08: Initiated bench-scale iodine capture studies using nonradioactive constituents

- **FY09: Continue iodine capture work:**
 - Use commercial sorbent IONEX Type Ag 900
 - Determine DFs under varying conditions (iodine concentration, co-constituents, temperature, residence times)
 - Dissolver off-gas may contain iodine at ~10 ppm
 - Combined vessel off-gas may contain iodine at ~10 ppb
 - Determine sorbent capacity





Summary of INL Off-Gas Work

- FY09: Install additional sorption columns for water vapor, CO₂ and Xe (ambient temperature operations)
 - An integrated bench-scale treatment unit
 - Currently plan to run with nonradioactive constituents
 - Establish baseline capture of water vapor, CO₂ and Xe on commercial adsorbents
 - Test alternative sorbents
 - Design Kr capture system, which may be added in future

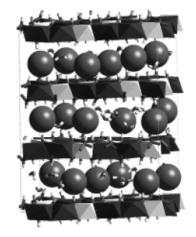




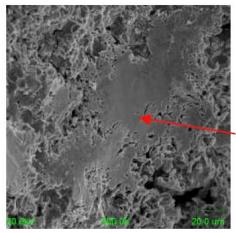
Encapsulants and Glasses as Iodine Waste Form

Bismuth-containing Encapsulants

- Two Trademark
 Applications for US Patents
 submitted;
 - 2007, 2008



Bi-I-O *in situ* layered oxides



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Low-Temperature Glasses for encapsulation of AgI, I₂ or AgI-MOR

- Glass transition temperatures lower than sublimation temperatures of AgI or I₂
- Glass compositions that enhance iodine retention

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Materials for Iodine Waste Streams - FY09 Plans at SNL

- Sorption of I₂ gas into traps for encapsulants
 - Low-temperature glasses
 - Alternative waste forms
- Primary focus will be on "standard" iodine loaded AgZ materials from ORNL and INL surrogate studies and CETE Hot tests
 - Retention of iodine
- Alternate sorbents or recovery operations to produce better waste forms





Off-gas Treatment Take-aways

- Multiple air emission regulations may apply to spent fuel reprocessing facilities
- Gaseous fission product control efficiencies up to 99.99% (DF = 10,000) may be required
- Technologies for controlling gaseous fission products are less established; new technologies and waste forms are under evaluation
- Ongoing R&D is continuing to determine performance, operating conditions, and waste forms for gaseous fission product control technologies



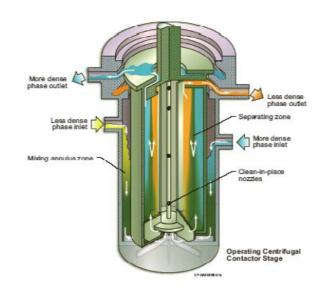


Role of Centrifugal Contactors in Aqueous Separations

- Centrifugal contactors will likely play a significant role in advanced aqueous recycle of used nuclear fuel
 - Utilized in portions of production scale facilities
 - Used for development of advanced separation processes
- Have several advantages over pulse columns and mixer-settlers
 - Less space required
 - High efficiency
 - Reach steady-state rapidly, allowing for rapid recovery from upsets and shutdown/restart
 - Reduced solvent contact times

But not without a few disadvantages

- Processes with slow kinetics
- Processes with likelihood of large quantities of solids (clean-in-place capability has potential to mitigate)
- Remote maintenance





Commercial Centrifugal Contactor Testing at the INL in 2008

- Testing of remotely operable/maintainable5-cm contactors
- Design of remotely operable/maintainable "production" scale 12.5-cm contactors



- Design and construction of 30 stage 5-cm contactor pilot plant
- Temperature profile testing in the newly constructed contactor pilot plant





Development of Remote 5-cm Centrifugal Contactor



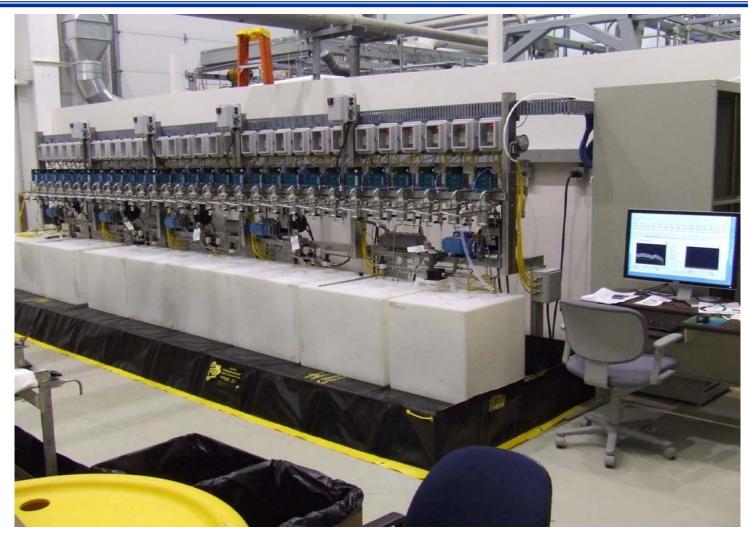








30 Stage 5-cm Centrifugal Contactor Pilot Plant





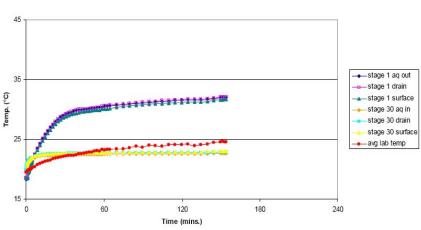


Test Objectives of Temperature Profile Testing in 30 Stage Pilot Plant

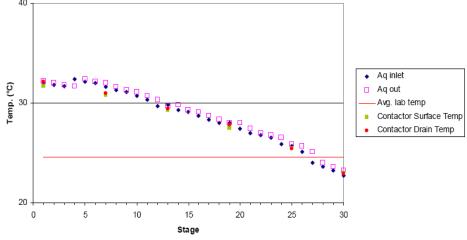
- Aqueous separation flowsheets typically require some level of temperature control – often different for various sections
- Laboratory scale centrifugal contactor testing (2-cm) results in large temperature increase for the process solutions due to heat generated from the motors and low flowrates
 - 2-cm centrifugal contactors with heat exchanger jackets were designed and utilized to alleviate this issue
- Will jacketed heat exchangers be required for engineering and production scale centrifugal contactors?
 - With larger flowrates it is expected that the temperature impact from motor heat will be reduced
 - Heating or cooling the process feed solutions may be enough to accomplish temperature control, preventing the need for a complex heat exchanger system for the centrifugal contactors
 - ANL has developed a computer model to predict process temperature based on system design. Limited experimental data is available - Data from testing will be used by ANL to validate/improve their model.



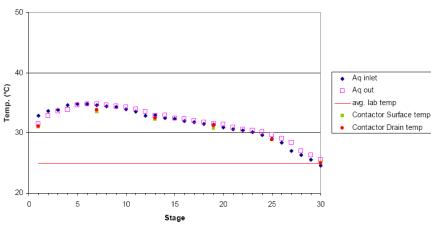
Temperature Profile Testing



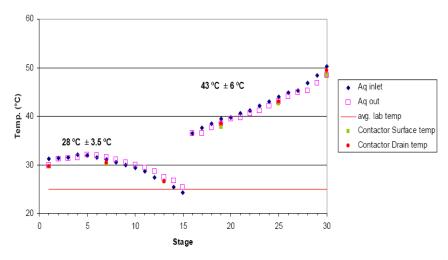
Aqueous Only - Approach to Steady State - Ambient Temp



Aqueous Only – Temperature Profile – Ambient Temp February 10, 2009



Two Phase Flow - Temperature Profile - Ambient Temp



Two Phase Flow - Temperature Profile - Strip at 50C





Summary of Results of Temperature Profile Tests

- Process solution temperature increased when feed solutions were at ambient temperature
- At feed temperatures of 50 °C, heat losses were greater than the heat gain due to motor operation or heat of mixing which resulted in processes solution temperature decreasing below the feed temperature
- Control of the feed solution temperature has a significant impact on process solution temperature
- Process temperature control in flowsheets using CINC 5-cm centrifugal contactors could likely be accomplished for many flowsheets by controlling the temperature of the feed solutions
- Processes that require tighter temperature control may require jacketed contactor or insulation





Testing Planned for FY 2009

- A prototype remotely maintainable 12.5-cm centrifugal contactor will be constructed and tested
- TRUEX mass transfer testing and temperature control testing will be performed using the 30 stage centrifugal contactor pilot plant
- A location and layout that will allow testing of the pilot plant with depleted uranium and/or low levels of radiotracers will be evaluated
- A pulse column will be set up for testing
 - Will be moved from the University of Idaho
 - Two inch glass pulse column that has approximately 20 feet of active height and upper/lower disengagement heads
 - Equipped with nozzle plates and an air pulser





Acknowledgements

ORNL

- J. Binder
- D. Ramey
- B. Spencer

SNL

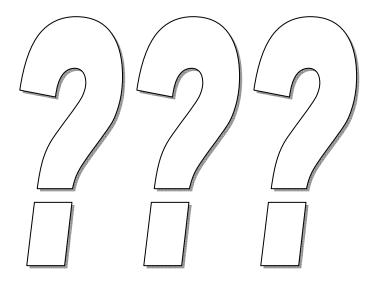
T. Nenoff

- D. Haefner
- J. Law
- T. Todd
- N. Soelberg

Prepared by Oak Ridge National Laboratory, P.O. Box 2008, Oak Ridge, Tennessee 37831-6285, managed by UT-Battelle, LLC, for the U.S. Department of Energy under contract DE-AC05-00OR22725.

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